

Study of fuel assemblies under severe accident top quenching conditions in the PARAMETER-SF test series

**(PARAMETER-SF3 and –SF4 experiments,
ISTC project proposal)**

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***CEG – SAM, 11th Meeting
Dresden, 7 -9 March 2007***

Objective:

The studying of behaviour of simulators fuel assembly VVER-1000 completed with standard constructional materials under severe accident conditions at the top flooding

Implementation:

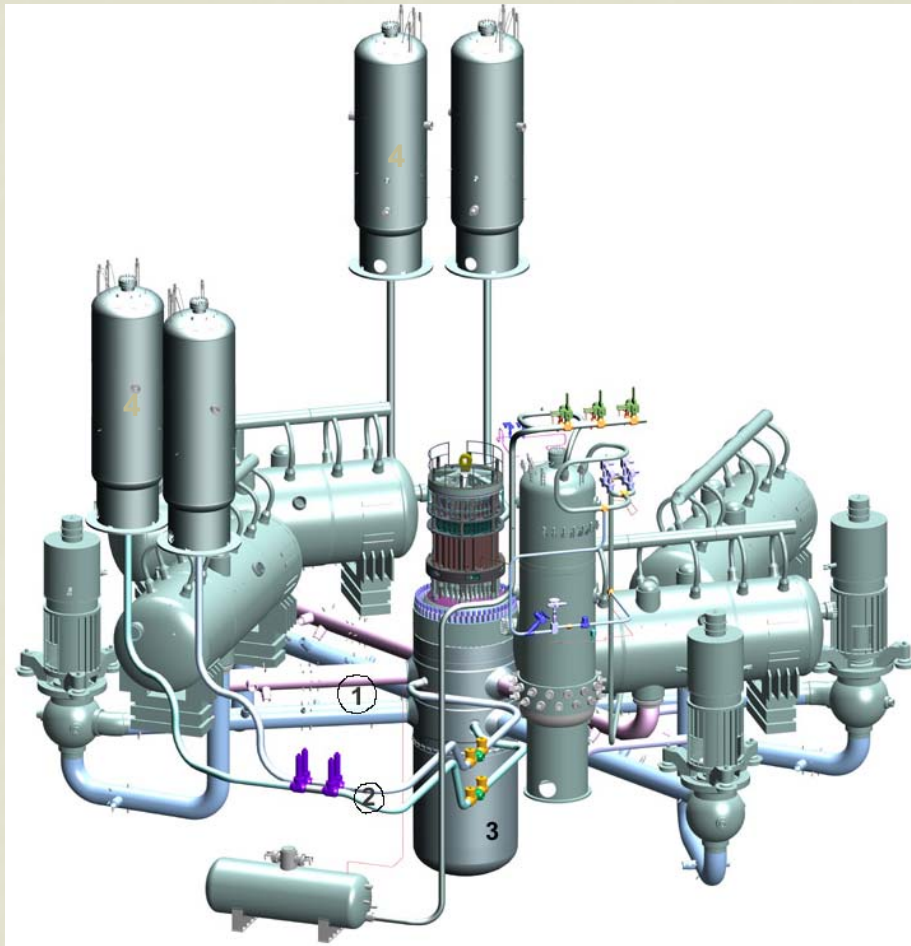
FSUE SRI SIA “LUCH” – rig experiments and post test analysis

IBRAE RAS – pre and post test analyses

FSUE EDO “GIDROPRESS” – justification of an experimental scenario, pre and post test analyses

Justification of test scenario

VVER-1000 3D view



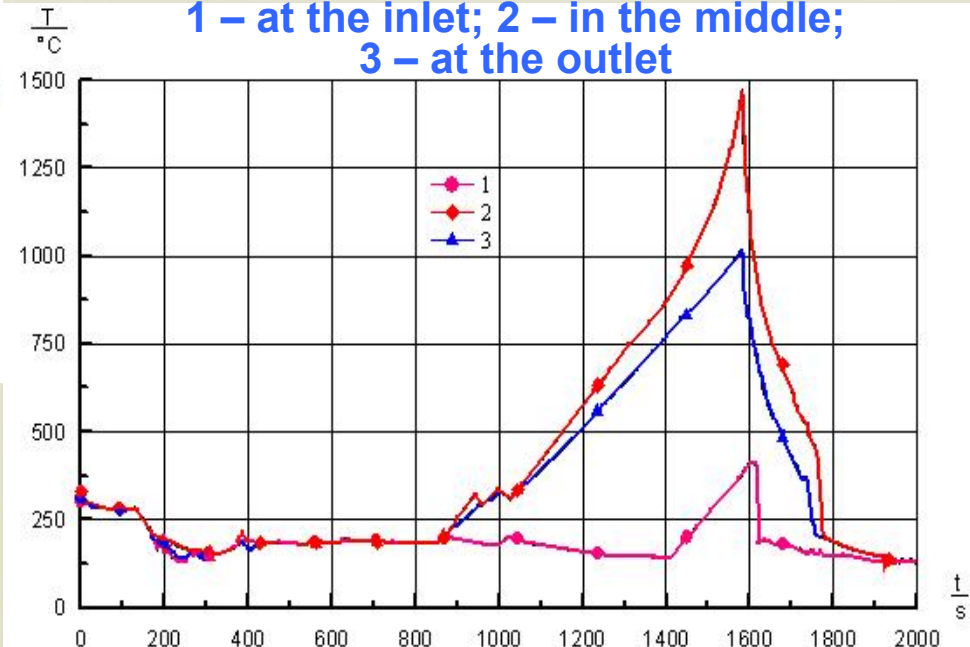
1 - главный циркуляционный трубопровод (холодная и горячая нитки одной петли);
2 - трубопроводы система аварийного охлаждения активной зоны;
3 - реактор; 4 - ёмкости САОЗ.

Simulated SA scenario of VVER-1000

- Large break of cold leg of Main Circulation Loop with fault of ECCS (ED 200);
- Recovery of 1 channel of ECCS at late accident stage in a range from 1600 up to 1800 C;
- Water injection from top and bottom of core with total rate 200 Kg/s

Clad temperature

1 – at the inlet; 2 – in the middle;
3 – at the outlet



Main outcomes of LBLOCA (ED 200) analysis

Design of WWER-1000 ECCS allows to provide simultaneous top/bottom flooding of overheated core

Pre flooding stage:

- Clad heat up rate – 1...2 K/s
- Steam flow rate – 0.2 g/s per rod

Flooding stage:

- Water injection – > 2 g/s per rod from top

At a simultaneous flooding of a core from top and bottom the top part of a core long time is exposed only to cooling by water from top at intensive movement of steam from bottom

Realization of the top flooding at SA

1. VVER - 1000

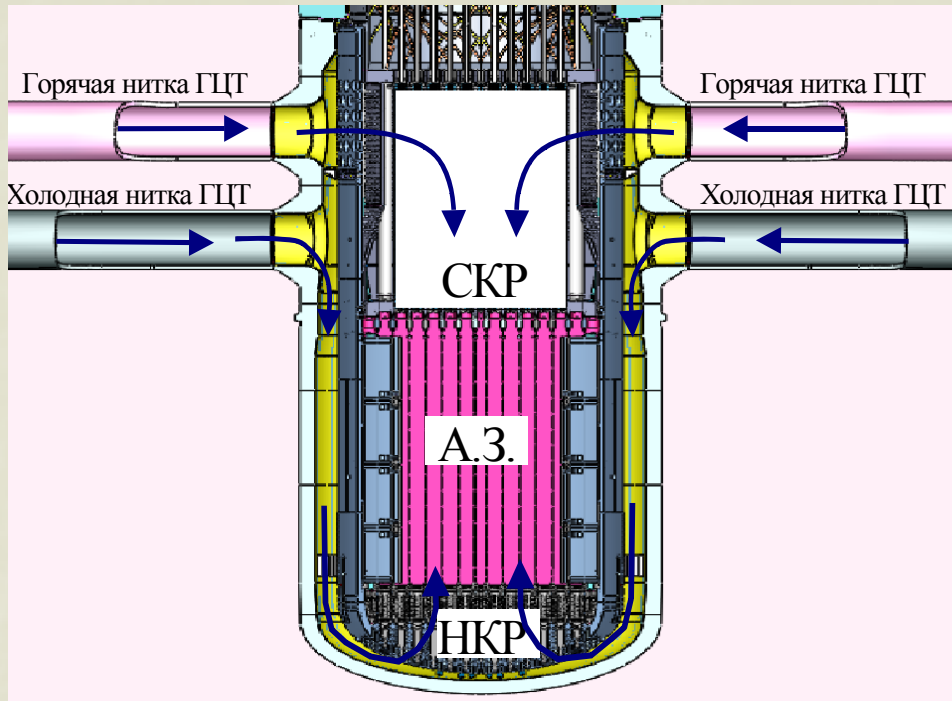


Схема залива активной зоны (А.З.) двумя насосами низкого давления:
-верхнего, через сборную камеру реактора (СКР);
-нижнего, через напорную камеру реактора (НКР).

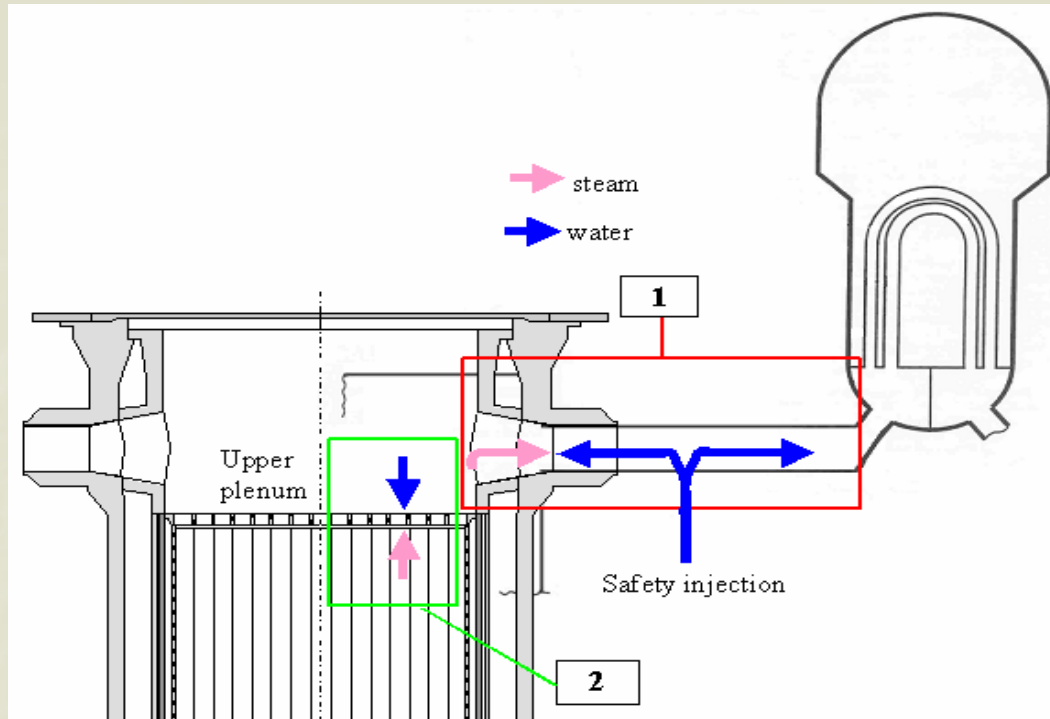
Cooling of a core it is determined by two processes:

1. Increase in a water level in the bottom chamber (at a top flooding and directly in the chamber at a bottom flooding).

2. Water inflow in the top chamber, distribution on section of a core and passage through a core from it cooling, filling of the bottom chamber with rise of a level of the coolant.

Water delivery from ECCS simultaneously from top and bottom allows to avoid a situation, when all submitted water is carried away in a leak. Water delivery from top allows to organize cooling earlier, than at water delivery from bottom.

2. PWR



There are two possibilities for top quenching occurrence:

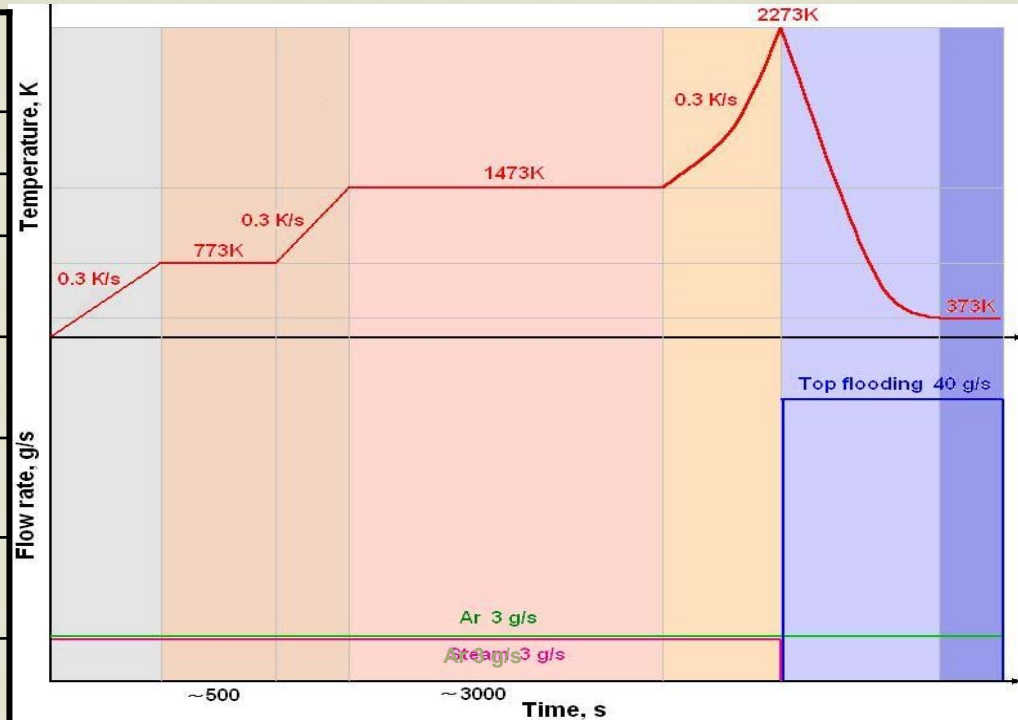
- 1. In case of severe accident, the steam in steam generator tubes can condensate and water thus produced can return to the core via the hot leg.**
- 2. When the injection point of safety injection system is located on the hot leg, one can expect that a part of the injected water rate can directly go to the core through the nozzle of the hot leg.**

The PARAMETER SF-1 Experiment

Main parameters of experiment

Coolant	Steam/ argon
Flow rate of coolant, g/s	3/2
Temperature of coolant, K	~773
Heating rate of cladding, K/s	0.3
Temperature of cladding at the pre-oxidation phase, K	1473K
Duration of the pre-oxidation phase, s	3000
Maximal temperature of cladding, K	2273
Quenching phase	Top flooding
Flow rate of top flooding, g/s	~ 40

Sequence diagram of experiment



Some conclusions by results of the analysis of PARAMETER SF-1 experiment

- 1. Difficult character of movement of cooling front of test bundle at a top flooding is caused by infringement of geometry of test bundle and blocking of through passage section of bundle by the formed zones of fusion.**
- 2. Process of degradation of constructional elements of bundle happened in experiment and absence of zones of destruction (debris) pellets are caused by presence of a skeleton of heaters of rods.**
- 3. The received results have shown on necessity of carrying out of the additional researches, allowing to estimate character of cooling of assembly at a top flooding before its degradation and in conditions of formation of zones of destruction pellets (debris).**

Experiment SF4

Research of moving of materials of the bundle VVER-1000 which have been heated up to temperature ~ 2070 K, as a result of partial destruction of rods and formation debris. Studying of influence of change of structure of assembly on cooling at the top flooding

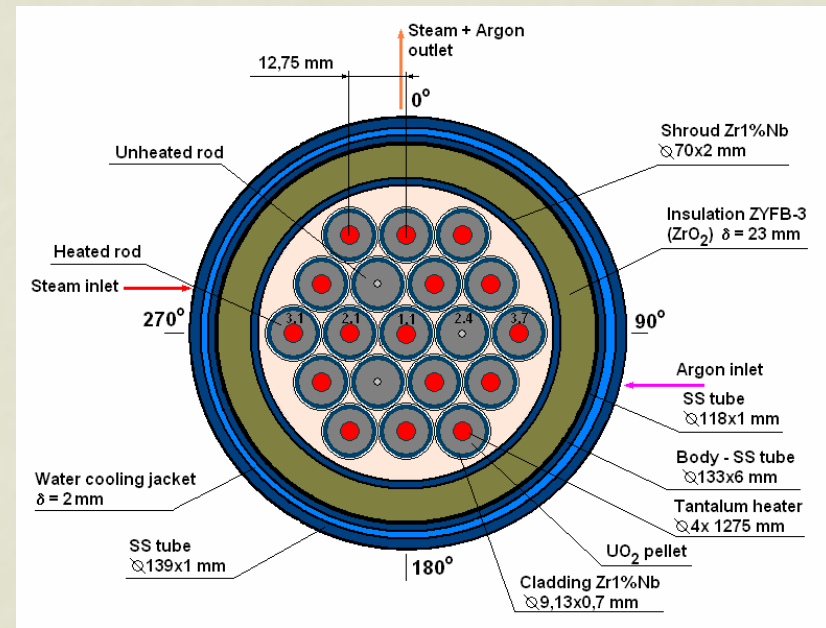
Object of tests:

19-rods test bundle VVER-1000 with 3 unheated rods

The main requirements of experiment:

The maximal temperature of bundle before top flooding – 2070 K

The water rate of top flooding - 40 g/s.



The prospective scenario of experiments

Stage	Stage substance	Main parameters			
		Bundle temperature, K	Medium	Heating/cooling rate	Time, s
1	Electric heating of the fuel bundle within argon	770	argon with flow rate of 3(2)* g/s	0.16	0-3000
2	Electric heating of the fuel bundle within the steam and argon flow	770	steam and argon with flow rate of 3/3(2) g/s	0	3000-5000
3	Heating of the fuel bundle (phase I)	770-1470	steam and argon with flow rate of 3/3(2) g/s	~0.3 K/s	5000- 8500
4	Pre-oxidation of the fuel bundle	~ 1470	steam and argon with flow rate of 3/3(2) g/s	0	8500-11500
5	Heating of the fuel bundle (phase II)	1470-1870(2070)**	steam and argon with flow rate of 3/3(2) g/s	0,1-0,3 K/s	Define experimentally as deign temperature will be reached
6	Flooding of the fuel bundle from the top (~5 s after Tmax reached)	up to saturation	water with flow rate of 40 g per bundle	100 – 10 K/s	~300

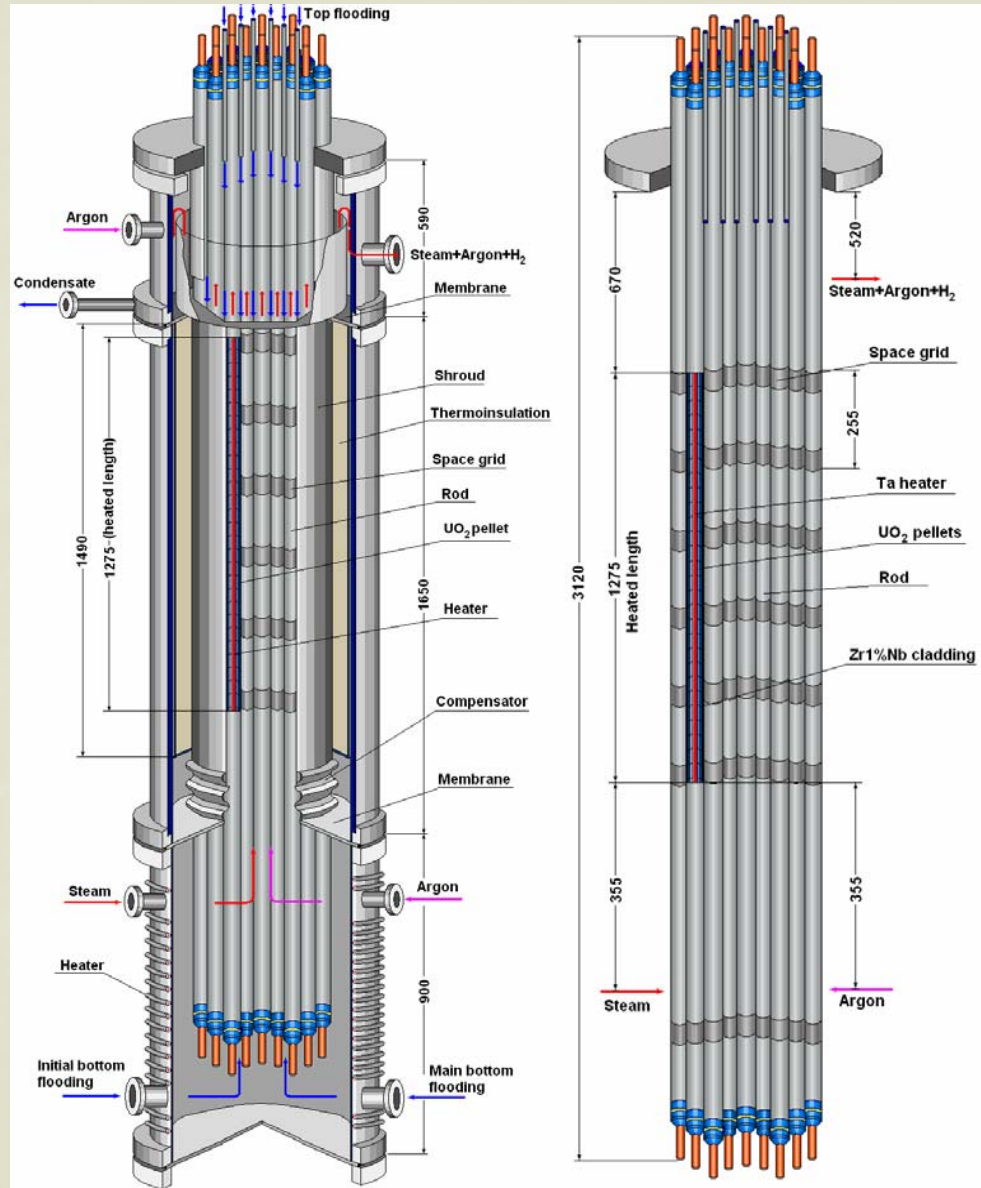
*) flow rate of argon of second test; **) Bundle temperature of second test.

Test Bundle

The main technical characteristics

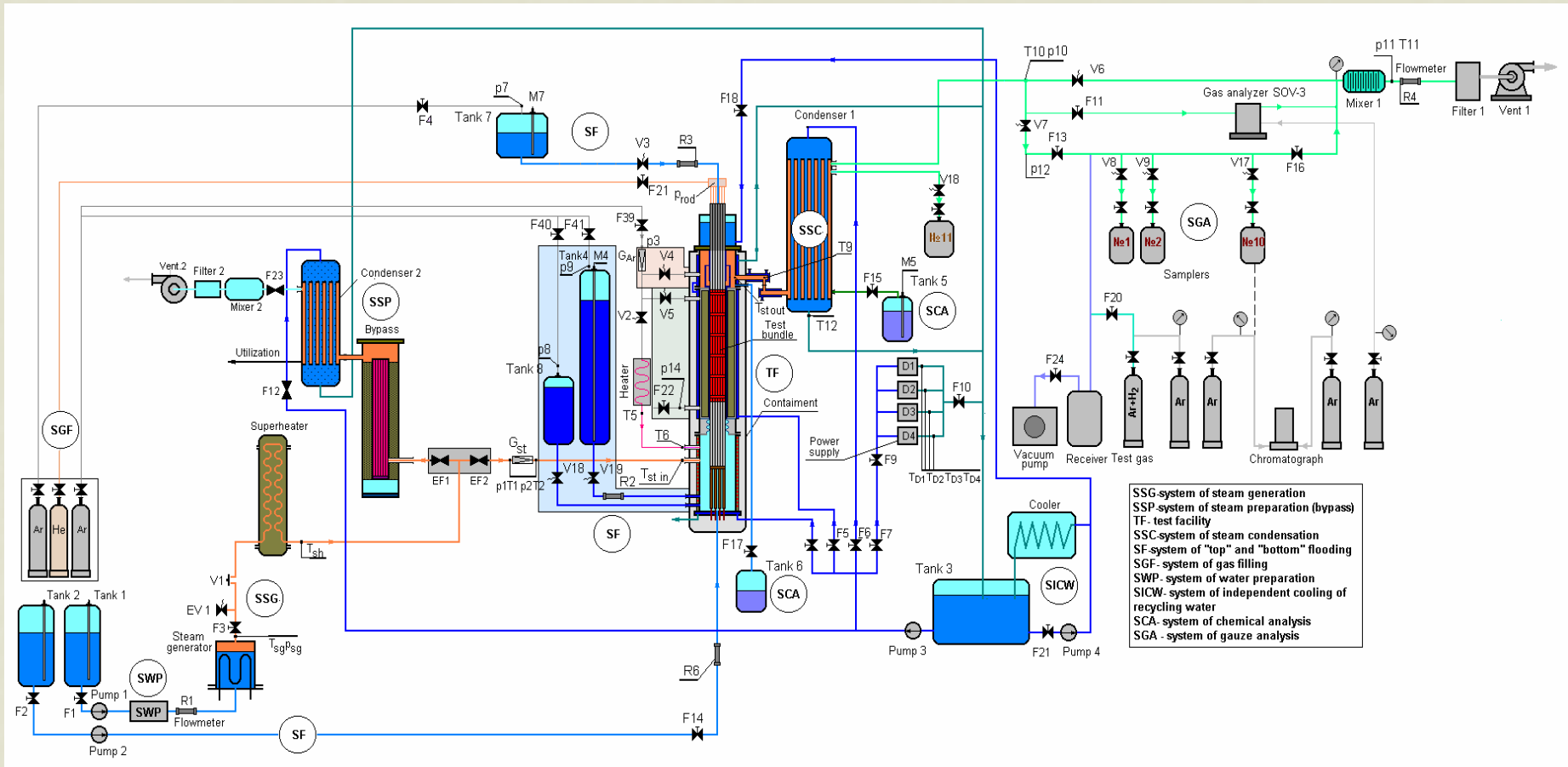
Type	VVER-1000
Number of rods	19
- heated	18
- unheated	1
Rods	
- cladding, mm	Ø 9,13/7,73 (Zr1%Nb)
- pellets	UO ₂
- heater	Ø 4/1275 (Ta)
Grid type	triangle
- grid pitch, mm	12.75
Spacing grid	Zr1%Nb
- height, mm	20
- spacing, mm	255
Shroud	Zr1%Nb
- thickness, mm	2
- diameter/height, mm	70/1490
Thermoinsulation	ZrO ₂ ZYFB-3
- thickness, mm	23
- diameter/height, mm	116/1490

General view



PARAMETER Facility

Functional diagram



Main technological systems

- System of separate input of argon;
- The monitoring system of balance of steam and water;
- System of top flooding;
- The monitoring system of hydrogen

Expected results of works under the Project

- Reception and ordering of the information on behaviour of model of fuel assembly VVER-1000 in severe accident conditions at the top flooding;
- Expansion of a database for verification SA codes (RELAP/SCADAPSKIM, MELCOR, ПАТЕГ-CБЕЧА, ATHLET, ICARE-CATHARE, etc.);
- Research of opportunities of a top flooding of fuel assembly, heated up to ~ 1870 K and ~ 2070 K;
- Use of the received data for increase safety of the projected and working atomic power stations with VVER and PWR.

The basic stages of the project

Tasks	Quarter 1	Quarter 2	Quarter 3	Quarter 4	Quarter 5	Quarter 6	Quarter 7	Quarter 8
Task 1	Post-test material analysis of the test bundle VVER-1000 PARAMETER SF-2 Experiment							
Task 2	Research of features of cooling and change of structure materials of the bundle VVER-1000 at temperature ~1870 K in conditions of the top flooding PARAMETER-SF3 Experiment							
Task 3					Research of moving of materials of the bundle VVER-1000 which have been heated up to temperature ~ 2070 K, as a result of partial destruction of rods and formations дебриса. Studying of influence of change of structure of assembly on cooling at the top flooding PARAMETER-SF4 Experiment			

Budget cost of the project

Full budget cost of the project (\$ USA)	600000
Payments to individual participants	335120
The equipment	100320
Materials	65560
Other direct costs	18000
Business trips	45000
Overhead charge	36000
Duration of the project (month)	24