



# International Science and Technology Center



## INVESTIGATION OF FISSION PRODUCT RELEASE FROM HIGH BURN-UP FUEL ANNEALED UNDER OXIDIZING AND REDUCTION CONDITION

**VERONIKA**

(**V**VER **E**xperiments on **R**elease due to **O**ver-heating:  
**N**ormalization and **K**nowledge **A**ugmentation).

**Project Proposal**

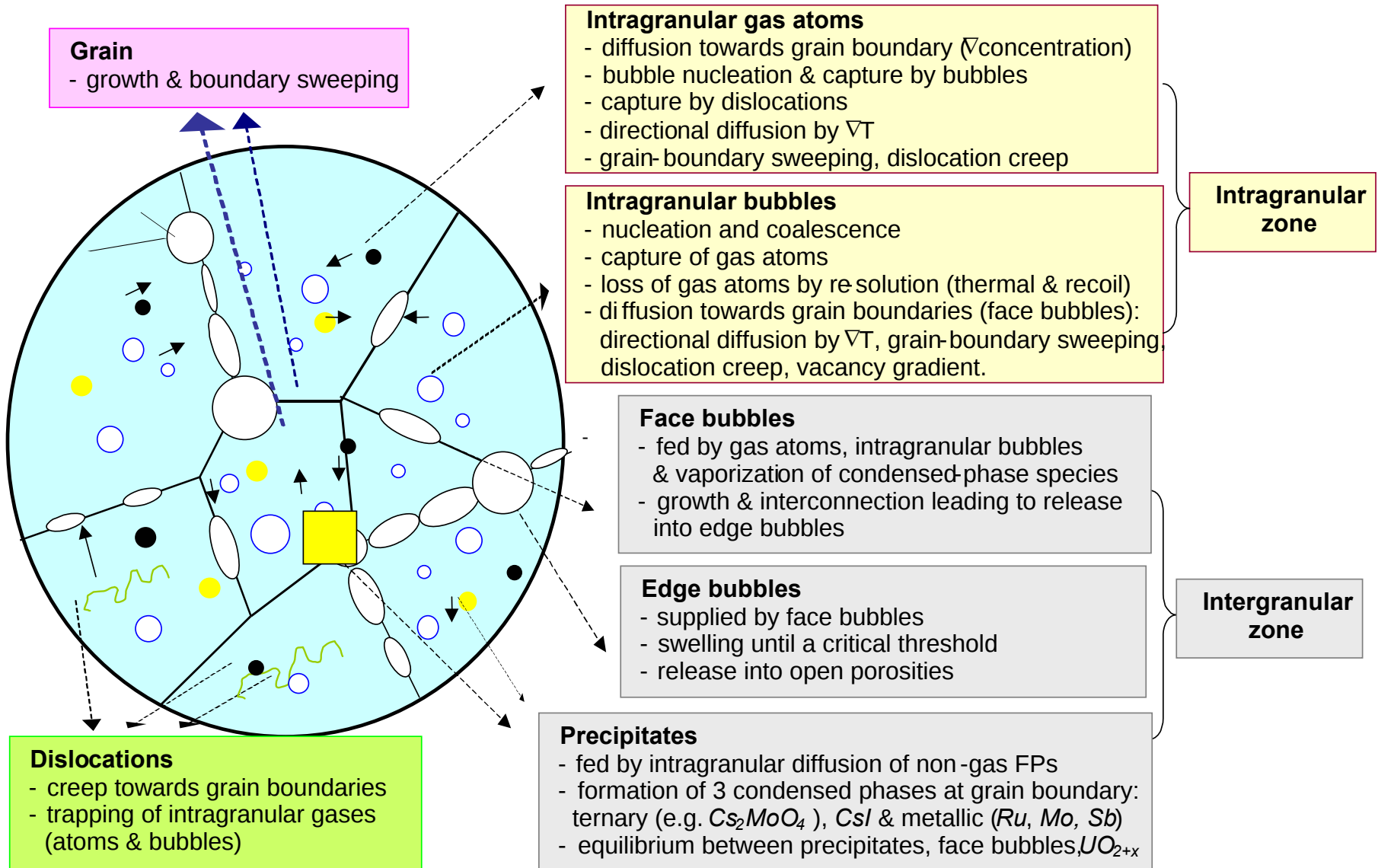
**RIAR (Dimitrovgrad), IBRAE (Moscow)**

**10<sup>th</sup> CEG-SAM Meeting**  
**Kurchatov-City, Republic of Kazakhstan**  
**IAE NNC, September 5-8, 2006**

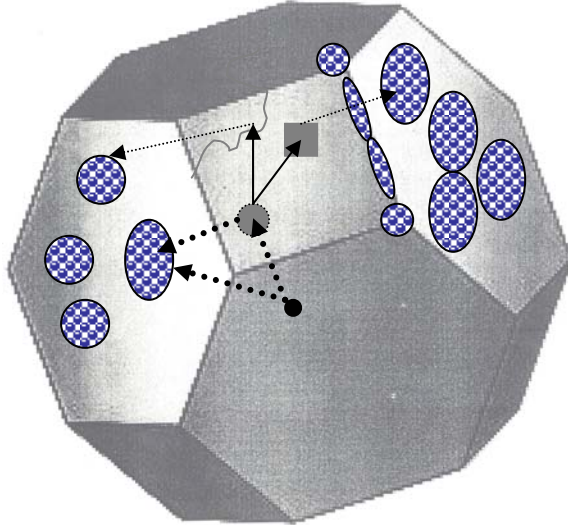
# Objectives of VERONIKA Project

- ***To obtain detailed experimental data on fission products release from highly irradiated VVER fuel along with fuel micro-structure evolution under severe accident conditions***
- ***To use these results for the development (and validation) of the physical models and numerical codes describing fuel behaviour and fission products release***

# MFPR Basic Models



# Transport of Fission Gases



System of equations:

$$\frac{\partial X_i}{\partial t} = F_i(X, t),$$

$$X = \{C_g, C_b, V_b, \rho_d, C_d, V_p, C_p, C_v, C_i\}$$

- Diffusion of gas atoms ( $C_g$ )
- Intragranular bubbles ( $C_b, N_b$ ) nucleation, growth and resolution (thermal and irradiation-induced)
- Intragranular bubbles diffusion (also biased in temperature gradient) and coalescence
- Interactions with dislocations ( $\rho_d$ ), sintering pores ( $C_p$ ), vacancies ( $C_v$ ) and interstitials ( $C_i$ )
- Intergranular bubbles formation, coalescence and interconnection...

# Scope of VERONIKA Project Tests

- *Investigation of fission products release from fuel with burnup of 60 MW\*d/kgU in oxidizing and reducing environments in the temperature range of 1400 - 2300°C*
- *Investigation of the release of a wide list of fission products including short living isotopes:  $^{85}\text{Kr}$ ,  $^{133}\text{Xe}$ ,  $^{131}\text{I}$ ,  $^{137}\text{Cs}$ ,  $^{134}\text{Cs}$ ,  $^{106}\text{Ru}$ ,  $^{103}\text{Ru}$ ,  $^{144}\text{Ce}$ ,  $^{99}\text{Mo}$ ,  $^{140}\text{Ba}$ ,  $^{95}\text{Zr}$  and other. That will be provided by pre-irradiation of the specimens in the research reactor*
- *Investigation of evolution of high burn-up fuel microstructure under tests conditions (by pre- and post-test microanalysis of samples)*

# Test Matrix

- *Two kind of specimens: bare fuel (BF) and fuel rod segments (RS) of about 12mm long (1 pellet).*
- *Three test temperatures – 1400, 1700, 2300 °C.*
- *Oxidizing and reducing environments.*

| Project stage               | Gas environment                                 | Flow rate, mg/s | Test temperature, °C |        |        |
|-----------------------------|-------------------------------------------------|-----------------|----------------------|--------|--------|
|                             |                                                 |                 | 1400                 | 1700   | 2300   |
| First stage<br>(3 years)    | H <sub>2</sub> /Ar                              | 0,45/5          | BF, RS               | BF, RS | BF, RS |
|                             | H <sub>2</sub> O                                | 25              | BF, RS               | -      | BF, RS |
| Second stage<br>(2.5 years) | (H <sub>2</sub> O/H <sub>2</sub> ) <sub>1</sub> | 25/0,45         | BF, RS               | BF, RS | BF, RS |
|                             | (H <sub>2</sub> O/H <sub>2</sub> ) <sub>2</sub> | 25/0,2          | BF, RS               | -      | BF, RS |

*Test matrix is aimed at 3 goals:*

- 1. To determine the influence of gas medium (H<sub>2</sub>O, (H<sub>2</sub>O/H<sub>2</sub>)<sub>1</sub>, (H<sub>2</sub>O/H<sub>2</sub>)<sub>2</sub>, H<sub>2</sub>/He) on FP release from specimens with and without cladding*
- 2. To determine the influence of temperature (1400, 1700 and 2300 °C) on FP release from specimens with and without cladding at the same gas-steam mixture content*
- 3. To determine the influence of fuel-cladding interaction in the reducing gas medium on FP release at temperatures of 1400, 1700 and 2300 °C*

## SARNET group comments to the test matrix and test procedure

|                                |                                                                                                                                                                                                                 |
|--------------------------------|-----------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------------|
| AEKI<br>EDF<br>IRSN<br>JRC/ITU | The test matrix should be extended to include air atmosphere so that the complete range of oxygen potentials likely to be encountered in a severe accident is covered: hydrogen, hydrogen/steam, steam and air. |
| IRSN                           | The oxidation state of the clad pre-test needs definition.                                                                                                                                                      |
| CEA<br>JRC/ITU                 | Technique for removing the clad minimising damage to the fuel outer rim zone needs to be defined and any damage sustained should be characterised.                                                              |
| CEA                            | The re-irradiation history needs attention too.                                                                                                                                                                 |
|                                | The fuel will need to be well characterised, e.g. regarding the grain size.                                                                                                                                     |
| IRSN                           | Gas flows and compositions in the case of mixtures, and exactly how and when the gas composition is to be changed (injection scenario).                                                                         |
| EDF                            | Also care needs to be taken in the choice of crucible material.                                                                                                                                                 |
| CEA<br>JRC/ITU                 | Use of SIMS and TEM.                                                                                                                                                                                            |
| CEA                            | Detailed point regarding collimator slits.                                                                                                                                                                      |
|                                | To perform two more tests (both with steel component, one with B <sub>4</sub> C and one without) in addition to their regular tests but without Niobium.                                                        |

# Revised test matrix

## 1 Stage

| Test | Sample type                                 | Environment         | Test temperature, °C |
|------|---------------------------------------------|---------------------|----------------------|
| 1    | Bare fuel                                   | H <sub>2</sub> / Ar | 1400                 |
| 2    | Bare fuel                                   | H <sub>2</sub> / Ar | 1700                 |
| 3    | Bare fuel                                   | H <sub>2</sub> / Ar | 2300                 |
| 4    | Fuel rod segment                            | Air / Ar            | 1400                 |
| 5    | Fuel rod segment                            | Air / Ar            | 1700                 |
| 6    | Fuel rod segment                            | Air / Ar            | 2300                 |
| 7    | Bare fuel                                   | Air / Ar            | 1400                 |
| 8    | Bare fuel                                   | Air / Ar            | 1700                 |
| 9    | Bare fuel                                   | Air / Ar            | 2300                 |
| 10   | Reference sample of initial irradiated fuel |                     |                      |

| Project stage               | Gas environment                 | Test temperature, °C |        |        |
|-----------------------------|---------------------------------|----------------------|--------|--------|
|                             |                                 | 1400                 | 1700   | 2300   |
| First stage<br>(3 years)    | H <sub>2</sub> /Ar              | BF                   | BF     | BF     |
|                             | Air/Ar                          | BF, RS               | BF, RS | BF, RS |
| Second stage<br>(2.5 years) | H <sub>2</sub> O/H <sub>2</sub> | BF, RS               | BF, RS | BF, RS |
|                             | H <sub>2</sub> O                | BF, RS               | -      | BF, RS |

# Main Test Procedures

- ***Manufacturing of the experimental rig***
- ***Initial fuel rod examinations***
- ***Refabricated fuel rods manufacturing***
- ***Refabricated fuel rods irradiation***
- ***Preparation and certification of fuel specimens (fuel pellets and fragments of fuel rods)***
- ***FP release tests***
- ***Post-test examination of fuel specimens***



# Initial fuel rod examination



## Objectives:

- *Selection of the typical spent fuel rod*
- *Avoid the cladding defects – debris, nodules etc*
- *Initial fuel and cladding certification*

## Test procedures

- *Eddy-current defectoscopy*
- *Y-scanning*
- *Profilometry*
- *Plenum gas analysis*

## Fuel and cladding structure characterization

**Metallography** – *Pellet crack pattern, fuel-cladding interaction/gap, fuel porosity, fuel grain size measurement.*

**SEM** - *(polished surface and fractography) porosity, intra- and intergranular bubbles pattern.*

**EPMA** – *Fission product distribution, precipitates composition.*

# Refabricated fuel rods manufacturing

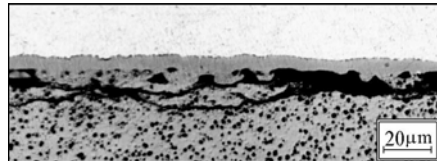
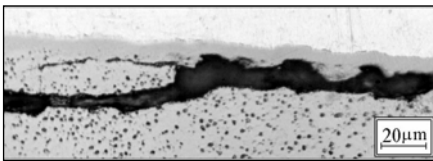
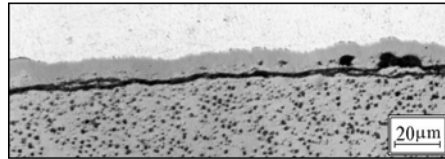
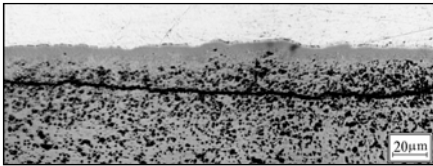
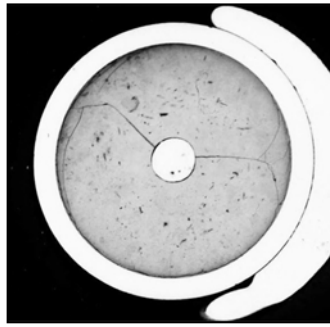
- Manufacturing of the short fuel rods (200 mm long) sealed by the welded end plugs and filled with He at 2-2.5 MPa from the fragments of initial fuel rod.

## Refabricated fuel rods irradiation

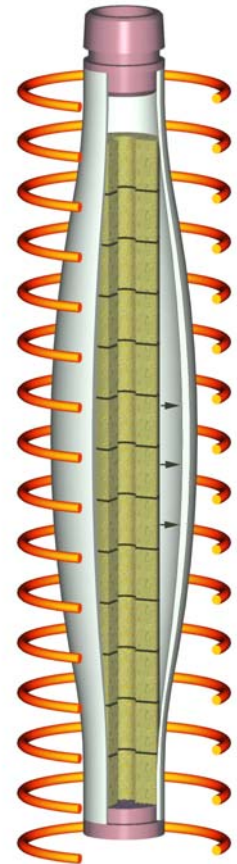
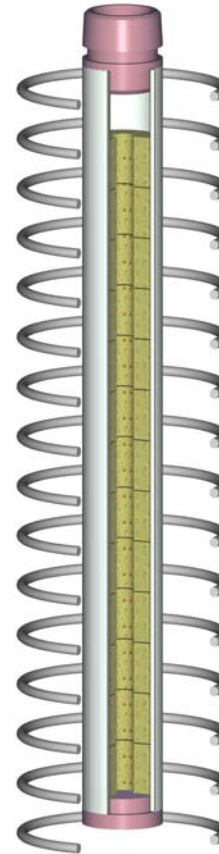
- Low power (30-50 Wt/cm)
- Low temperature (to be defined in pre-test calculations)

**Objective:** accumulation of the short-lived isotopes in the solid solution; prevent any fuel structure transformation

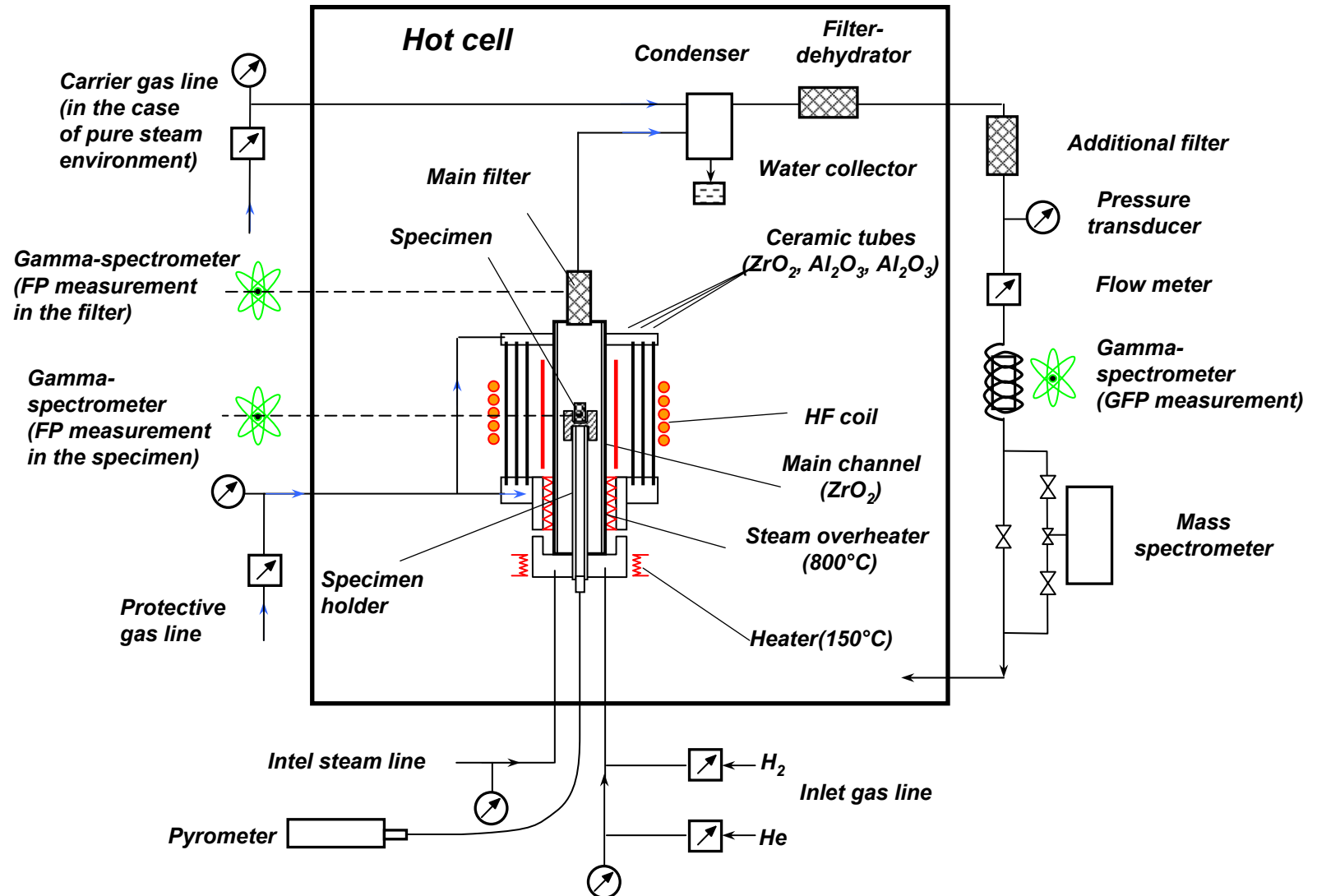
### *Radial and contraction cracks in the fuel*



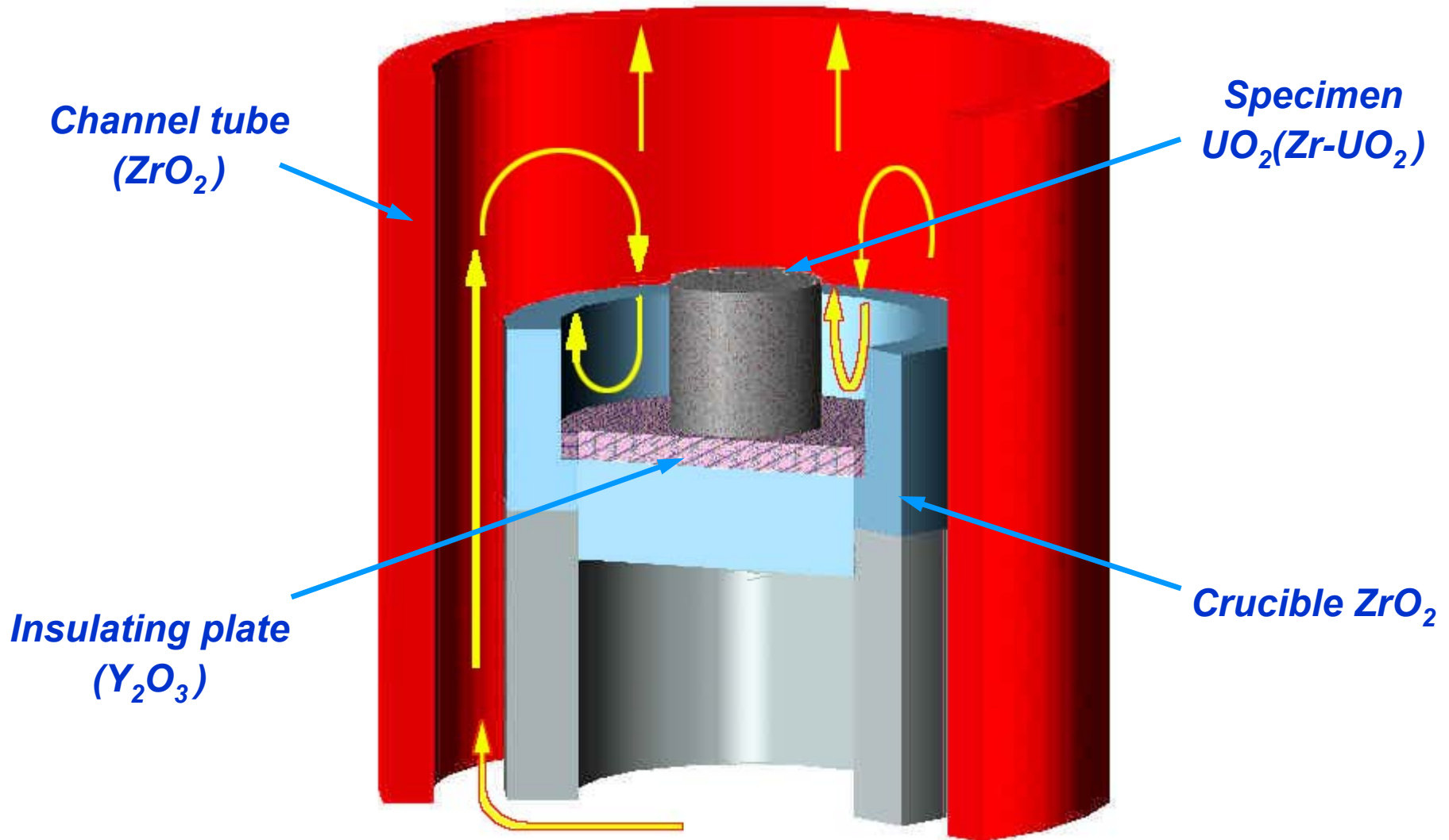
### *Pellet-clad separation by heating the pressurised rods up to 700°C*



# Test Rig

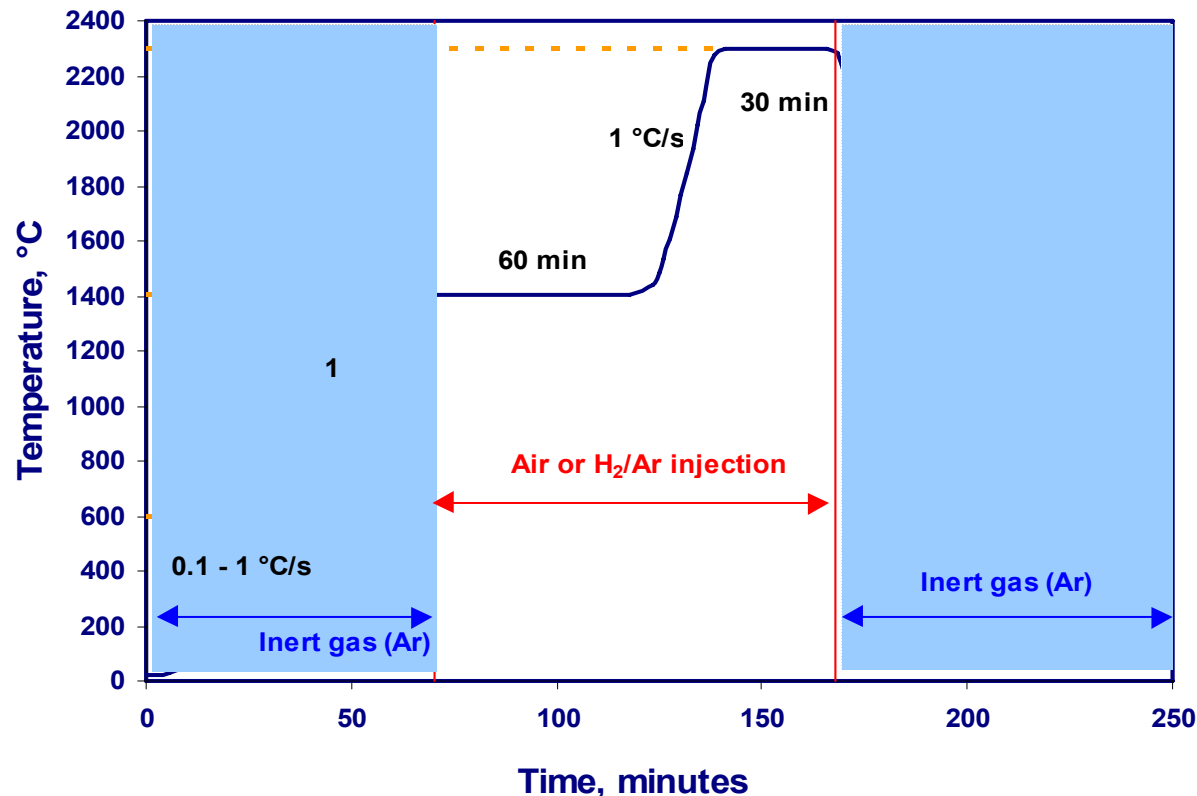


# Crucible materials



## Test temperature

- heating in the inert atmosphere up to the temperature of 600 °C
- heating from 600 up to 1400 °C with a rate of 1 K/s
- annealing at 1400 °C during 1 hour (complete clad oxidation)
- heating with a rate of ~ 1 K/s up to the specified temperature (1700 or 2300 °C)
- annealing during 30 minutes
- end of heating



# Test Performance

## ***Parameters to be measured during the test***

- *intensity of GFP in the delay gas coil*
- *intensity of FP in a fuel specimen*
- *intensity of FP on the main filter*
- *hydrogen concentration in the carrier-gas at outlet*

## ***Results to be obtained***

- *relative FP release*
- *hydrogen generation (in the case of steam injection)*

# Post-Test Examinations

## Fuel and cladding structure characterization

### Optical metallography

- *grain size*
- *porosity*
- *gas swelling*
- *Fuel – cladding interaction*
- *Selection the positions for SEM / EPMA examinations*

**SEM** - *(polished surface and fractography) porosity, intra- and intergranular bubbles pattern.*

**EPMA** – *Fission product distribution, precipitates composition.*

**TEM** – *is not available*

**SIMS** – *May be applied to the very restricted number of specimens. Just the qualitative results are expected*

## Part B. MFPR (Model for Eission Products Release)

### Objectives

***On the base of new experimental results:***

- to develop theoretical models of fission products and irradiated VVER fuel behaviour under conditions of severe accidents***
- to improve and to adapt physical models and codes developed for PWR fuel to VVER fuel***

- *Pre-test calculations of new experiments for determination of parameters and conditions of the tests*
- *Processing and analysis of results of new experiments*
- *Development and improvement of the physical models on fission products release and high burnup VVER fuel behaviour under conditions of severe accident*
- *Implementation of the developed and improved models in the MFPR code*
- *Verification of the MFPR code against the new experimental database and other new available data at the end or during the program*

## **The first stage**

**(3 years duration, 777 400 \$) :**

- **$T_1 0$  – beginning of the work under the project;**
- **$T_1 0+1$  year – manufacturing and testing of the experimental rig, 315 k\$;**
- **$T_1 0+1$  year - adaptation of the MFPR code to the new experimental rig, pre-test calculations, 26.2 k\$;**
- **$T_1 0+2,5$  years – first series of tests (10 tests), 346.2 k\$;**
- **$T_1 0+3,0$  years – theoretical analysis of the obtained experimental data, development of models and codes, 80 k\$.**

## **The second stage**

**(2.5 years duration, 494 000 \$):**

- **$T_2 0$  – beginning of the second stage work;**
- **$T_2 0+1$  year – second series of tests (10 tests), 343 k\$;**
- **$T_2 0+2.5$  years –post-test specimen examinations; theoretical analysis of the obtained experimental data, development of models and codes, 151 k\$.**

**The estimated total cost of the Project (2 stages) – 1 217 400 \$**